Influence of convective heat losses on the H-mode power threshold


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In order to trigger the transition to the high confinement mode (H-mode), the power lost through the separatrix into the scrape-off layer (SOL), $P_L$, estimated as the sum of the Ohmic and auxiliary heating power from which the time derivative of the plasma stored energy is substracted, should exceed some threshold value $P_{th}$. A simple scaling for $P_{th}$ has been deduced from the experimental data collected on several tokamaks with divertors [1],

$$P_{th} = 0.042 \bar{n}_e^{0.64} B^{0.78} S^{0.94},$$

where $\bar{n}_e$ is the line averaged electron density in $10^{20} m^{-3}$, $S$ is the plasma surface area in $m^2$, and $B$ is the toroidal magnetic field in T.

At the same time, there are many experimental situations where the conditions for the H-mode onset can not be described by the Eq.(1) [2-4]. In the present contribution the mechanisms of such deviations will be analyzed by means of predictive transport modelling for the cases of the L-H transition at low plasma densities and in limiter configurations. In both situations the power required to establish the H-mode is significantly larger than the one predicted by the multi-machine scaling (1). As an example, Fig. 1 shows the variation of $P_L$ normalized to the value predicted by Eq.(1) with the edge density measured in JET discharges with MKIIGB septum divertor [4]. When the density is below a limit value of $1.3 \times 10^{13}$ cm$^{-3}$, the H-mode power threshold exceeds the scaling prediction by a factor up to 2.

The existence of the minimum in the L-H power threshold at low density is a phenomenon observed in many tokamaks. The low density limit blow which the power threshold increases with decreasing density can be related to the condition when the penetration depth of neutrals recycling from divertor plates is of the order of $0.1*\kappa\alpha$, where $\kappa$ is the elongation.
and $a$ is the plasma minor radius [5]. This constraint can be rewritten a condition on the plasma density at the edge: 

$$n_e [cm^{-3}] > n_{crit} - 10^{15} (\kappa a [cm])^{-1}.$$  (2)

If $n_e < n_{crit}$ neutrals penetrate freely into the confined plasma volume. The same takes place in a limiter configuration where the distance, which neutrals have to pass in the SOL, is normally much smaller than in a divertor one.

On the basis of numerical computations with 1-D transport code RITM [6], in the present contribution we interpret the interrelation between the neutral penetration into the confined plasma and the increase of the power threshold for the L-H transition above the value prescribed by the Eq.(1). With the present transport model [7-9], which includes contributions from the most important core and edge micro-instabilities, the code allows to reproduce self-consistently the formation of the edge transport barrier (ETB) if the total heating power exceeds some critical level.

In addition to radiation, the energy is lost from the plasma through two channels: due to heat conduction proportional to the temperature gradient, $-\nabla T$, and with convection associated with the particle flow, $-D_n \nabla n$. Here $\chi$ and $D$ are the heat and particle diffusion coefficients. At the last closed magnetic surface (LCMS), the fraction of the convective loss in the total is given by the expression: 

$$Q_{conv} / Q_{tot} = \left(1 + \frac{\chi}{3D} \frac{\delta_n}{\delta_T} \right)^{-1},$$

where $\delta_n = n / \nabla n$ and $\delta_T = T / \nabla T$ are the density and the temperature e-folding lengths. The prescribed values of $\delta_n$ and $\delta_T$ taken normally from experimental data are imposed in RITM computations as boundary conditions at the LCMS. Thus, by varying the ratio $\delta_n / \delta_T$, $Q_{conv} / Q_{tot}$ can be changed from 0 to 1.

Figure 2 compares the power threshold for the L-H transition obtained in computations (black symbols) with the one given by the scaling law (white symbols), for different values of the $Q_{conv} / Q_{tot}$ ratio obtained by varying $\delta_n$ (points) and $\delta_T$ (boxes), respectively. At a low fraction of convective heat losses, the computed power coincides with the scaling predictions. An increase of $Q_{conv} / Q_{tot}$ above 50% leads to a strong increase of the computed threshold power with respect to the scaling predictions.
This can be interpreted as follows. The transport model used in RITM computations [7,8] prescribes that under the L-mode conditions the anomalous transport at the plasma edge is mostly due to drift Alfven unstable modes driven by collisions and current perturbations. These modes are stabilized when the pressure gradient, driven mostly by the temperature gradient, increases and the plasma collisionality reduces when the heating power grows up. An increase of the convective channel for heat losses results in a reduction of the temperature and its gradient. This prohibits the suppression of the turbulent transport and postpones the L to H-mode transition to a higher heating power.

Normally, the change of the dominant mechanism for the heat losses from conductive to convective takes place by switching from a divertor to a limiter configuration, when the distance between the confined plasma and plasma facing components increases, or by the reduction of the density below some critical value. In both situations the ratio of the distance travelled by neutrals in the SOL, \(d_{\text{SOL}}\), to their mean free path length before ionization decreases. Therefore a larger fraction of neutrals, recycling from the limiter surface or divertor plate, penetrate inside the LCMS, the charged particle flux in the confined region increases and the heat losses associated with particle convection enhance. This can be demonstrated by using the improved two-point model [10]. Figure 3 shows the variation of the \(Q_{\text{conv}}/Q_{\text{tot}}\) ratio with the edge density computed for two different values of the distance travelled by neutrals in the SOL before entering the confined plasma, \(d_{\text{SOL}}=10\) cm and \(d_{\text{SOL}}=30\) cm.

\[
\frac{\Gamma_{\text{CMS}}}{\Gamma_{\text{CMS}}} = \exp(-n\sigma_{\text{LCMS}}d_{\text{SOL}}),
\]

with \(\sigma_{\text{LCMS}}\) being the effective cross-section for the neutral losses due to ionization and charge-exchange. In a steady state, the influx of neutrals should be balanced by the outflow of charged particles, \(D_{\text{n}} n/\delta_{n}\). Thus, the reduction of the edge density or \(d_{\text{SOL}}\) would lead to the stronger density gradient or shorter \(\delta_{n}\). For the given heat source, the increase of convective losses, caused by stronger particle flux, leads to a corresponding reduction of the conductive heat flux component. The latter leads to the reduction of the temperature and its gradient, and, as it is mentioned above, hinders the ETB formation.

The radial profiles of the density scale-length for two values of the edge density, measured by the lithium beam diagnostic, for two JET discharges from the sequence plotted in fig.1, are shown in fig.4. The discharge with the higher edge density (solid curve) is characterized.
by larger $\delta_n$ in the vicinity of the separatrix. The reduction of density results in the steeping of the density profile (dashed curve) and, therefore, in increase of heat losses due to charged particle convection. Unfortunately, for extremely low density, where the measured power deviates from the scaling predictions, no data from the lithium beam diagnostic are available. Such measurements are a subject for further experiments. Nevertheless, the tendency shown in fig.4 is in agreement with that obtained from improved two-point model, see fig.3.

**Summary**

Calculations done with 1-D transport code RITM show the strong dependence of the L-H threshold power on the dominant mechanism for the heat losses at the edge. Whereas, in the situation when the fraction of the convective heat losses in the total one does not exceed 50%, the computed threshold power coincides with the scaling prediction. An increase of heat losses due to charged particle convection leads to the increase of the threshold power by several times compared to the multi-machine scaling. Under these conditions, corresponding to the low density discharges or to the limiter configuration, the heating power required to establish the ETB is up to a few MW larger than it is predicted by the scaling.

The analysis done with two-point model demonstrated that, the relative contributions of convective and conductive heat losses are determined by the transport of neutrals recycling from neutralizing plates and, in particular, by the ratio of the distance which they should pass in the SOL before entering the confined plasma to their mean free path. The decrease of the edge density or the distance traveled by neutrals before entering the confined plasma, which occurs in the switch from divertor to limiter tokamak, leads to a larger fraction of particles penetrating through the LCMS and, therefore, changes the heat flux balance to the convection dominated one.

Measurements of edge profiles in JET discharges with MKIIGB septum divertor demonstrate that, the discharges with lower density are characterized by steeper edge density profile and, therefore, by stronger heat losses due to charged particle convection, in agreement with analysis done with improved two point model.

**References**


